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Plasma performance improvements with liquid lithium limiters in CDX-U

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Abstract

The use of flowing liquid lithium as a first wall for a reactor has potentially attractive physics and engineering features. The current drive experiment-upgrade (CDX-U) at the Princeton Plasma Physics Laboratory has begun experiments with a fully toroidal liquid lithium limiter. CDX-U is a compact ($R = 34$ cm, $a = 22$ cm, $B_{\text{toroidal}} = 2$ kG, $I_P = 100$ kA, $T_e(0) \sim 100$ eV, $n_e(0) \sim 5 \times 10^{19} \text{ m}^{-3}$) short-pulse (< 25 ms) spherical tokamak with extensive diagnostics. The limiter, which consists of a shallow circular stainless steel tray of radius 34 cm and width 10 cm, can be filled with lithium to a depth of a few millimeters, and forms the lower limiting surface for the discharge. Heating elements beneath the tray are used to liquefy the lithium prior to the experiment. The total area of the tray is approximately 2000 cm². The tokamak edge plasma, when operated in contact with the lithium-filled tray, shows evidence of reduced impurities and recycling. The reduction in recycling and impurities is largest when the lithium is liquefied by heating to 250 °C. Discharges which are limited by the liquid lithium tray show evidence of performance enhancement. Radiated power is reduced and there is spectroscopic evidence for increases in the core electron temperature. Furthermore, the use of a liquid lithium limiter reduces the need for conditioning discharges prior to high current operation. The future development path for liquid lithium limiter systems in CDX-U is also discussed.

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1. Introduction

The use of liquid lithium as a plasma facing component (PFC) is presently undergoing tests at

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the PISCES-B divertor simulator facility [1], the T11-M tokamak [2], and the current drive experiment-upgrade (CDX-U) at the Princeton Plasma Physics Laboratory [3]. CDX-U is a spherical tokamak with major radius $R_0 = 34$ cm, minor radius $a = 22$ cm, a toroidal magnetic field of 2 kG, peak plasma current of less than 100 kA, central electron temperature $T_e(0) \sim 100$ eV, central electron density $n_e(0) \sim 5 \times 10^{19} \text{ m}^{-3}$, with a modest pulse duration of 25 ms or less. The experiment has extensive spectroscopic and other diagnostics to determine edge and core concentrations of lithium and other impurities. The CDX-U experiments described here focus on the use of large-area, free surface, liquid lithium PFCs.

The first experiments with lithium systems in CDX-U utilized a rail limiter [1,4] with a lithium-wet mesh, a system which is somewhat similar to the T11-M capillary limiter [2]. The rail limiter experiments were intended to explore the deployment of a small liquid lithium system in a tokamak. In that sense, the experiments were successful, since the limiter was installed, operated, and removed without problem. The lithium rail limiter did not, however, produce a significant effect on plasma performance. Following the completion of the rail limiter experiments, a circular tray 34 cm in radius, 10 cm wide, and 0.5 cm deep was mounted on the bottom of the CDX-U vacuum vessel and loaded with lithium. A view of the interior of CDX-U with the tray installed, prior to pumpdown, is shown in Fig. 1. If the tray is uniformly filled with liquid lithium, it presents an area of 2000 cm^2 to the plasma.

The tray limiter is fitted with resistive disk heaters clamped to the lower surface, which are capable of heating the tray to $400 \text{ }^\circ\text{C}$, well in excess of the melting point of lithium ($180 \text{ }^\circ\text{C}$). The tray was typically heated to $250 \text{ }^\circ\text{C}$ to liquefy the lithium. During operation of CDX-U, the tray serves as a fully toroidal limiter for the discharge, and hence forms a principal PFC for CDX-U.

However, the tray has not been uniformly wet by the lithium, so that perhaps 50% of the tray is uncoated stainless steel. The thickness of the lithium layer in the tray varies from 0 to 8–9 mm. The lithium exhibits varying degrees of coating, which were only partially removed by exten-

sive argon glow discharge cleaning. An unforeseen consequence of the insulating coatings acquired by the lithium in the tray was that unipolar arcing to the lithium was common during initial operation with the limiter. $\mathbf{J} \times \mathbf{B}$ forces produced during the short-lived arcs ejected small droplets of lithium from the tray. The droplets formed an irregular coating on the lower heat shields in the CDX-U vacuum chamber, but had no apparent consequences for tokamak performance. Despite the coatings, and the uneven layer of lithium filling the tray, there are strong indications of plasma performance improvements through the use of liquid lithium PFCs.

Results of the experiments on CDX-U with a liquid lithium rail limiter have been reported elsewhere [1,3,4]. Additional results on the implementation of the toroidal liquid lithium limiter, edge plasma interactions, and the effects of lithium wall coatings have also been reported [3,5,6]. Here we are primarily concerned with the effects of the liquid lithium limiter on plasma performance.

2. Edge plasma characteristics with the liquid lithium limiter

During operation with the tray limiter the edge oxygen, D_α , carbon and lithium emission was monitored by a spectroscopy system developed at the Oak Ridge National Laboratory. The system views on the order of 100 cm^2 of tray area from above. The intensity of emission at the time of peak plasma current was then extracted, for discharges which used the empty stainless steel tray as a limiter, for discharges which used a room-temperature lithium-filled tray as a limiter, and for discharges in which the tray was heated to $250 \text{ }^\circ\text{C}$, well above the melting point of lithium. The data for OII emission at 4416 \AA is shown in Fig. 2. Operation with solid lithium in the tray results in lowered edge oxygen emission than operation with the bare stainless steel tray. Liquefying the lithium results in a further reduction in edge oxygen. The data for the three tray conditions is segregated; the oxygen emission for the liquid lithium case is lowest, followed by the solid lithium and then the bare tray data. There is a clear trend toward

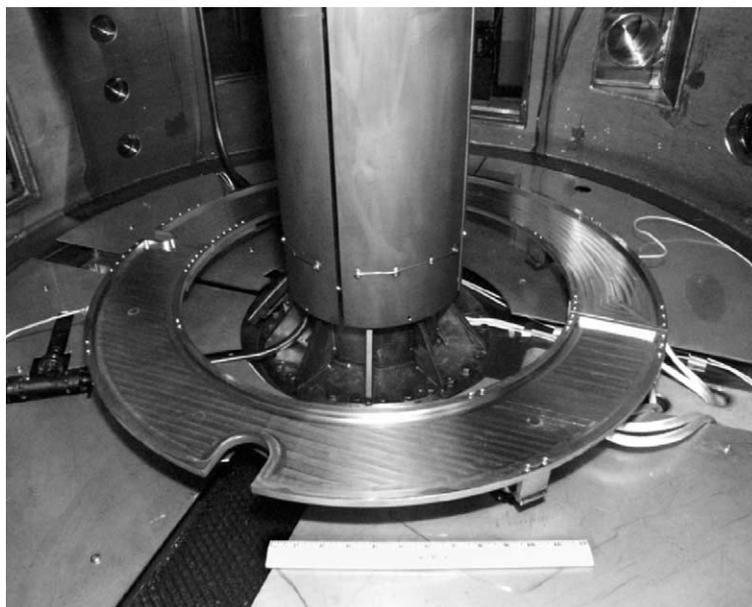


Fig. 1. Photograph of the interior of CDX-U showing the tray limiter, centerstack heat shields, and lower vessel shield plates.

higher plasma current as the oxygen emission drops. The limiter and fill were argon-glow discharge cleaned for several hours prior to obtaining the data sets for all tray fill conditions. Note also that the data for discharges limited on liquid lithium was taken prior to the data with a solid lithium limiter, so that the improvement in performance with liquid lithium cannot be ascribed to a gradual day-to-day conditioning effect.

Fig. 3 is a plot of the edge carbon emission from the CIII line at 4650 Å for the cases of a solid and a liquid lithium fill in the tray. CIII data is not

available for discharges limited by the bare stainless steel tray. There is a trend toward lower carbon in discharges with higher peak operating current, but the edge carbon emission is roughly comparable for both the liquid and solid lithium cases, for similar plasma current.

Fig. 4 is a plot of D_{α} emission in the edge plasma at the tray, for solid and liquid lithium. D_{α} emission data for the bare tray is not available. Virtually all the discharges with liquid lithium in the tray show reduced D_{α} emission compared to the discharges with solid lithium. This result agrees with previously obtained data indicating that the

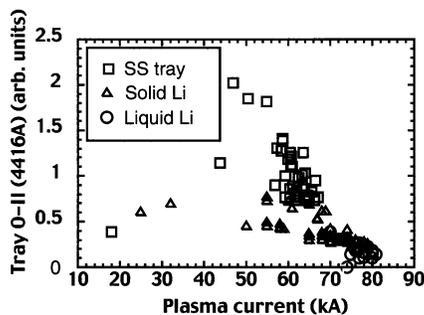


Fig. 2. Oxygen II emission at the surface of the tray, for discharges limited by the empty tray, a solid lithium fill, and a liquid lithium fill.

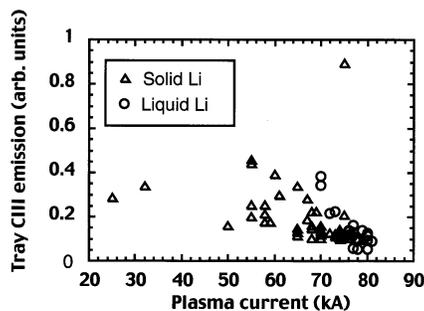


Fig. 3. CIII emission at 4650 Å from the edge plasma adjacent to the lithium tray limiter, for solid and liquid lithium.

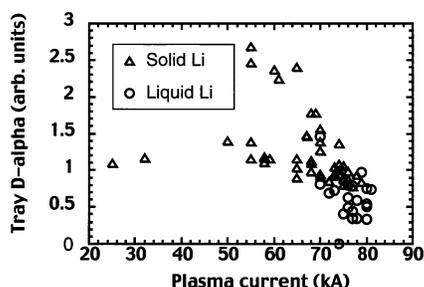


Fig. 4. D_{α} emission at the lithium-filled tray, for solid and liquid lithium limited discharges.

recycling coefficient for hydrogenic species on liquid lithium is very low [1]. Hydrogen implanted in the liquid lithium is able to diffuse rapidly out of the implantation zone into the bulk liquid metal [7], whereas hydrogen implanted in solid lithium saturates the surface. It should also be stressed that the viewed area represented in Fig. 4 is partly covered with lithium, and some of the lithium is oxide coated. Therefore, the residual recycling seen in the data with liquid lithium may be due to the contribution from a small bare area on the tray, or a coated area on the lithium fill. Reduced D_{α} emission is also correlated with increased plasma current.

3. Core plasma characteristics with the liquid lithium limiter

The area of the tray which is filled with lithium represents approximately 20% of the total PFC area in contact with the plasma, in a typical CDX-U discharge. The centerstack constitutes the principle remaining PFC. Despite the fact that the lithium in the tray represents a small fraction of the total plasma facing area, enhancements in global plasma performance have been observed in operation with the liquid lithium limiter. One indication of enhanced global performance was that a general reduction in the total radiated power was seen for discharges operated in contact with either solid or liquid lithium, as shown in Fig. 5.

For the electron temperature range in which CDX-U operates (~ 100 eV), the level of soft X-ray emission from carbon V is a good indicator of the electron temperature. Carbon V emission for

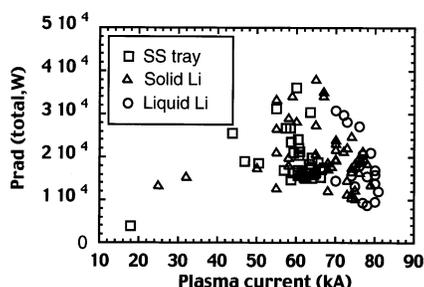


Fig. 5. Total radiated power, from the CDX-U tangential bolometer array, for discharges limited by the bare stainless steel tray, solid lithium, and liquid lithium.

the cases of a bare, solid lithium-filled, and liquid lithium-filled tray is shown in Fig. 6. Discharges limited by solid lithium show somewhat higher CV emission (and hence higher core electron temperatures) than the discharges limited by the bare stainless steel tray. However, the majority of discharges limited by liquid lithium show significant increases in the level of CV emission over those limited either by the bare tray or solid lithium. For discharges in which the carbon concentration does not vary significantly, as shown by the edge carbon data shown in Fig. 4, this increase in emission is indicative of higher core electron temperatures.

Somewhat higher peak plasma currents were also obtained in discharges limited by liquid lithium [5,6]. Since CDX-U is operated at constant loop voltage rather than feedback controlled for constant plasma current, higher plasma currents

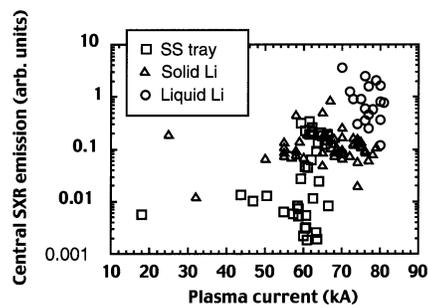


Fig. 6. Soft X-ray (CV) emission from CDX-U, for discharges limited by the bare stainless steel tray, solid lithium, and liquid lithium. The increase for the case of liquid lithium indicates a significant (perhaps $2-3\times$) increase in the peak electron temperature, in comparison to the bare tray and solid lithium cases.

are also indicative of cleaner, hotter plasmas. Loop voltage consumption is also slightly lower for plasmas limited on liquid lithium than for plasmas limited on stainless steel or solid lithium, although the magnitude of the reduction is small.

Lithium III emission from the core plasma at 135 Å is also monitored with a multichord array, provided by the spectroscopy group at Johns Hopkins University. The emission data, at the time of peak ohmic current, is shown in Fig. 7. Data from the bare tray provides a baseline. LiIII emission from plasmas limited on solid and liquid lithium is comparable, and does not show a dependence on plasma current. A quantitative estimate of the core lithium concentration from this data is not yet available.

4. Discussion

The implementation of large-area liquid lithium limiters in CDX-U required a significant technical effort. Further development of methods to produce large-area, clean liquid lithium surfaces in contact with the edge plasma is required. Techniques to restrain the liquid metal and prevent motion due to plasma-induced $\mathbf{J} \times \mathbf{B}$ motion are needed. However, it is clear from the experiments performed to date that a tokamak plasma can be successfully operated with liquid lithium PFCs. A drop in recycling and especially impurities results, which reduces radiated power from the discharge and increases the core electron temperature. Finally, recovery and cleanup of the interior of

CDX-U after lithium operations has proven to be straightforward.

In the near term, the existing tray limiter will be replaced. New filling and discharge cleaning techniques developed in collaboration with the PISCES group at the University of California at San Diego will be implemented, in an attempt to obtain a uniform lithium fill of the new tray, with a clean surface. Following experiments with the new tray and fill system, an internally circulating liquid lithium limiter will be installed in CDX-U next year. Experiments will be expanded to include the induction of $\mathbf{J} \times \mathbf{B}$ and ponderomotive forces to restrain the liquid metal during a discharge.

In CDX-U, most plasma material interactions take place at the centerstack which is coated with titanium nitride. In the longer term, it is desirable to eliminate all plasma surface interactions with materials other than liquefied lithium, in order to determine the effect of very low recycling on the confined plasma. A proposal to install a conformal shell in CDX-U, along with an electron beam evaporation system capable of depositing several hundred angstroms of lithium over the entire inner surface of the shell between discharges, has been made.

Acknowledgements

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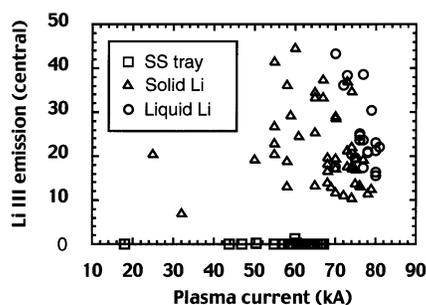


Fig. 7. Lithium III emission at 135 Å for plasmas limited by the bare tray (baseline data), solid lithium, and liquid lithium.